

NUCLEAR REGULATORY COMMISSION

[Docket No. 50-259; License No. DPR-33]

Tennessee Valley Authority; Notice of Informal 10 CFR 2.206 Public Hearing

The U.S. Nuclear Regulatory Commission (NRC) will hold an informal public hearing regarding a petition submitted pursuant to 10 CFR 2.206 involving Browns Ferry Nuclear Plant, Unit 1, of the Tennessee Valley Authority (TVA or the licensee). The hearing will be held on October 26, 1998. The location of the hearing will be at the Browns Ferry Nuclear Plant Training Center, Auditorium. The Browns Ferry Nuclear Plant Training Center is located at Shaw Road and Nuclear Plant Road, Athens, Alabama. The hearing will be open to public attendance and will be transcribed.

The structure of the hearing shall be as follows:

Monday October 26, 1998:

1:00 p.m.—NRC opening remarks
1:15 p.m.—Petitioner's presentation
2:00 p.m.—NRC questions
2:15 p.m.—Licensee's presentation
3:00 p.m.—NRC questions
3:15 p.m.—Public Comments
3:45 p.m.—Licensee/Petitioner's final statements
4:00 p.m.—Meeting concludes

By letter dated April 5, 1997, the Union of Concerned Scientists (UCS or Petitioner) submitted a Petition pursuant to 10 CFR 2.206 requesting (1) that the operating license for Browns Ferry Nuclear Plant, Unit 1 be revoked and (2) that the NRC require TVA to submit either a decommissioning plan or a lay-up plan for Browns Ferry Nuclear Plant, Unit 1. In addition, the Petitioner requested a hearing on this petition to present new information on Browns Ferry Nuclear Plant, Unit 1 that would include a discussion of the licensing basis reconstitution that would be required to support restart, and certain financial aspects that might be a consideration for the TVA's decision for retaining the Browns Ferry Unit 1 operating license.

The purpose of this informal public hearing is to obtain additional information from the Petitioner, the licensee, and the public for NRC staff use in evaluating the Petition. Therefore, this informal public hearing will be limited to information relevant to issues raised in the Petition. The staff will not offer any preliminary views on its evaluation of the Petition. The informal public hearing will be chaired by a senior NRC official who will limit presentations to the above subject.

The format of the informal public hearing will be as follows: opening remarks by the NRC regarding the general 10 CFR 2.206 process, the purpose of the informal public hearing, and a brief summary of the Petition and its Addendum (15 minutes); time for the Petitioner to articulate the basis of the Petition (45 minutes); time for the NRC to ask the Petitioner questions for purposes of clarification (15 minutes); time for the licensee to address the issues raised in the Petition (45 minutes); time for the NRC to ask the licensee questions for purposes of clarification (15 minutes); time for public comments relative to the Petition (30 minutes); and time for licensee and Petitioner's final statements (15 minutes).

Members of the public who are interested in presenting information relative to the Petition should notify the NRC official named below, 5 working days prior to the hearing. A brief summary of the information to be presented and the time requested should be provided in order to make appropriate arrangements. Time allotted for presentations by members of the public will be determined based upon the number of requests received and will be announced at the beginning of the hearing. The order for public presentations will be on a first received first to speak basis.

Written statements will also be accepted and included in the record of the hearing. Written statements should be mailed to the U.S. Nuclear Regulatory Commission, Mail stop O-14B21, Attn: Albert W. De Agazio, Washington, DC 20555.

Requests for the opportunity to present information can be made by contacting Albert W. De Agazio, Project Manager, Division of Reactor Projects-I/II (telephone 301-415-1443) between 8:00 a.m. to 5:45 p.m. (EDT), Monday-Friday. Persons planning to attend this informal public hearing are urged to contact the above 1 or 2 days prior to the informal public hearing to be advised of any changes that may have occurred.

Dated at Rockville, Maryland, this 22nd day of September 1998.

For the Nuclear Regulatory Commission.

John A. Zwolinski,

*Acting Director, Division of Reactor Projects—
I/II, Office of Nuclear Reactor Regulation.*
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NUCLEAR REGULATORY COMMISSION

[Docket Nos. 50-369 and 50-370]

Duke Energy Corporation; McGuire Nuclear Station, Units 1 and 2; Environmental Assessment and Finding of No Significant Impact

The U.S. Nuclear Regulatory Commission (the Commission) is considering issuance of amendments to Facility Operating License Nos. NPF-9 and NPF-17 issued to Duke Energy Corporation (DEC or the licensee) for operation of the McGuire Nuclear Station, Units 1 and 2 (McGuire), respectively, located at the licensee's site in Mecklenburg County, North Carolina.

Environmental Assessment*Identification of the Proposed Action*

The proposed action will replace the McGuire current Technical Specifications (CTS) to be consistent with the Improved Standard Technical Specifications (ITS) based on Revision 1 to NUREG-1431, "Standard Technical Specifications Westinghouse Plants BWR/4" April 1995, and the CTS for McGuire Units 1 and 2. The proposed action is in response to the licensee's application dated May 27, 1997, as supplemented on March 9, March 20, April 20, June 3, June 24, July 7, July 21, July 22, August 5, September 8, and September 15, 1998.

The Need for the Proposed Action

It has been recognized that nuclear safety in all plants would benefit from improvement and standardization of the TSs. The Commission's "NRC Interim Policy Statement on Technical Specification Improvements for Nuclear Power Reactors" (52 FR 3788, February 6, 1987), and later the Commission's "Final Policy Statement on Technical Specification Improvements for Nuclear Power Reactors" (58 FR 39132, July 22, 1993), formalized this need. To facilitate the development of individual improved TSs, each reactor vendor owners group (OG) and the NRC staff developed standard TSs (STS). For Westinghouse plants, the STS are published as NUREG-1431, and this document was the basis for the new McGuire Unit 1 and Unit 2 TSs. The NRC Committee to Review Generic Requirements reviewed the STS and made note of the safety merits of the STS and indicated its support of conversion to the STS by operating plants.